

$$0 + 1 \rightarrow 2 + 3$$

$$r_{01} = N_0 N_1 \int_0^{\infty} v P(v) \sigma(v) dv \equiv N_0 N_1 \langle \sigma v \rangle_{01}$$

$$\langle \sigma v \rangle_{01} = \left(\frac{8}{\pi m_{01}} \right)^{1/2} \frac{1}{(kT)^{3/2}} \int_0^{\infty} E \sigma(E) e^{-E/kT} dE$$

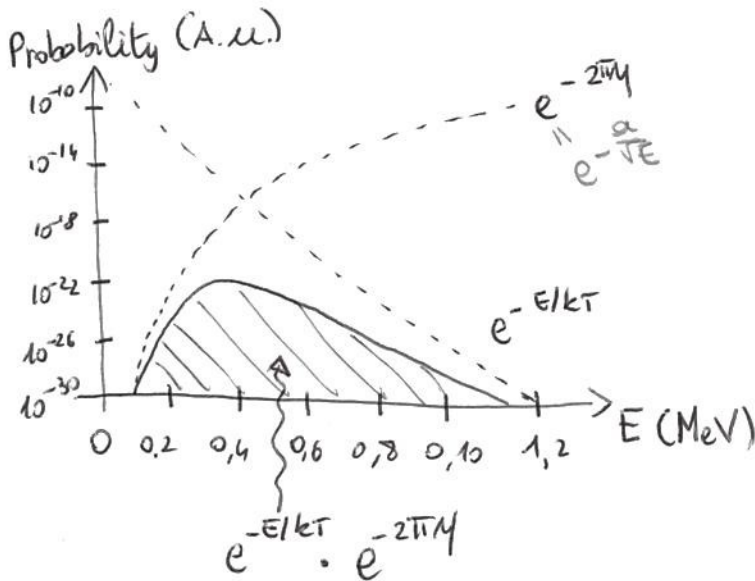
$$N_A \langle \sigma v \rangle_{01} = \left(\frac{8}{\pi m_{01}} \right)^{1/2} \frac{N_A}{(kT)^{3/2}} \int_0^{\infty} E \underbrace{\sigma(E)} e^{-E/kT} dE$$

$$\sigma(E) \equiv \frac{1}{E} e^{-2\pi\eta} S(E)$$

$$= \left(\frac{8}{\pi m_{01}} \right)^{1/2} \frac{N_A}{(kT)^{3/2}} \int_0^{\infty} \underbrace{e^{-\frac{\alpha}{\sqrt{E}}} e^{-2\pi\eta}}_{\text{GAMOW PEAK}} e^{-E/kT} dE$$

GAMOW PEAK
"

Energy range over which most nuclear reactions occur in the plasma



CONTROLLED FUSION REACTORS

In order to have controlled fusion reactions and reasonable energy, the thermonuclear fuel have to be heated up to 10^8 K (mean particle $E_k \approx 10$ keV) while it is kept to a high density long enough to generate ^{desired} power.

At these Temperature, atoms must become ionized (for H only 13.6 eV is needed to strip the electron); the fuel is a hot mixture of positive ions and negative electrons, but overall is electrically neutral. This is called PLASMA. The electrostatic properties of a plasma determine a length scale called DEBYE LENGTH

$$L_D = \left(\frac{4\pi\epsilon_0 k_B T}{e^2 n} \right)^{1/2}$$

≡ Measure of the charge carriers' net electrostatic effect in a solution & how far its electrostatic effect persists.

Where n is the average density of ion or electrons.

Using solid densities (10^{28} cm^{-3}), the Debye length for a 10 keV plasma is of the order of 10^{-8} m and the number of ptc in a volume of the plasma of dimension 1 Debye length is $\approx 10^4$.

For a more rarified plasma ($\rho \approx 10^{22} \text{ m}^{-3}$), $L_D = 10^{-5} \text{ m}$ and $n_D = 10^7$

In either cases, there are 2 basic properties: the physical size of the reacting plasma is far larger than Debye length in dimension, and there are many ptc in any characteristic volume.

These 2 properties permit to use plasma equations to describe thermonuclear fuel.

A major problem is to CONFINE the plasma: hot fuel exchanging energy with walls will simultaneously cool the fuel and melt the container.

2 schemes are used to confine thermonuclear fuel: a) magnetic confinement

b) inertial confinement. (solid pellet heated and compressed by being struck simultaneously from many directions with intense beams of γ or ptc)

Confinement of plasma is not absolute, since there are some ways ~~in~~ ^{for} the plasma to lose energy.

Primary mechanism is bremsstrahlung (since there ~~are~~ ^{are} electrons accelerated in a magnetic field), in which Coulomb scattering of two ptc produces an acceleration in turn which give rise to emission of

radiation.

The largest accelerations are suffered by lighter ptc, but since electrons and ions are in thermal equilibrium, any loss by electrons is felt also by ions, which will be less energetic and less successful in penetrating Coulomb barrier.

The power radiated by an electron experiencing acceleration a is: (120)

$$P = \frac{e^2 a^2}{6\pi\epsilon_0 c^3}$$

If r is the distance between ion and e^-

$$a = \frac{F}{M_e} = \frac{ze^2}{4\pi\epsilon_0 M_e r^2}$$

If τ is the characteristic time in which ion and electron interacts, the number of ions encountered at a distance r is:

$$n (v_e \tau) 2\pi r dr, \quad n \equiv \text{density of ions}$$

$$\Rightarrow dP = \frac{e^2 n}{6\pi\epsilon_0 c^3} \frac{z^2 e^4 (2\pi r dr) v_e \tau}{(4\pi\epsilon_0)^2 m_e^2 r^4}$$

dP = contribution to the total power from electrons scattered at impact factor r and $r+dr$.

$$\tau = \frac{r}{v_e} \Rightarrow dP = \frac{4\pi e^6 n}{3(4\pi\epsilon_0)^3 m_e^2 c^3} \frac{dr}{r^2}$$

Integrated from r_{\min} and r_{\max} dP , gives the power radiated by a single electron and multiplied by n_e gives the power per unit time radiated by plasma. $r_{\max} \sim \infty$; $r_{\min} \equiv$ distance of closest approach.

For a 10 keV electron; $r_{\min} = 144 Z \text{ fm}$.

The Q.M. uncertainty for such electron, and taking $\Delta p \sim p = 100 \text{ keV}/c$

$$\Delta x \approx 2000 \text{ fm} \left[\Delta p \Delta x = \hbar \quad \Delta x = \frac{\hbar}{\Delta p} = \frac{\hbar c}{\Delta p} = \frac{200 \text{ MeV} \cdot \text{fm}}{100 \text{ keV}} = \frac{2 \cdot 10^8}{10^5} = 2000 \text{ fm} \right]$$

$$\Delta x = \frac{200 \text{ MeV} \cdot \text{fm}}{10 \text{ keV}} = 2$$

r_{\min} cannot be as precise as $144 Z \text{ fm}$ (which is $\ll \Delta x$) and r_{\min} is better to be expressed as: $r_{\min} = \frac{\hbar}{m_e v_e}$. After the integration:

$$P = \int_{\frac{\hbar}{m_e v_e}}^{\infty} \frac{4\pi e^6 z^2 n n_e}{3(4\pi\epsilon_0)^3 m_e^2 c^3} \frac{dr}{r^2}$$

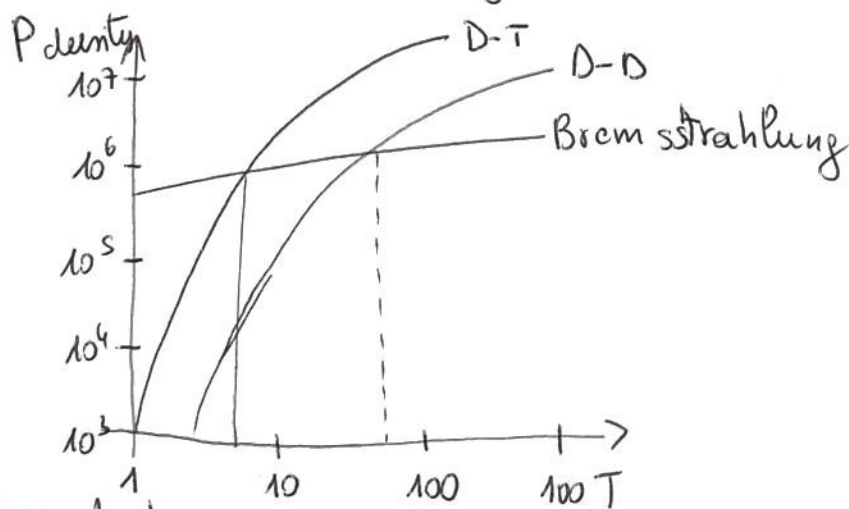
$$= \frac{4\pi n n_e z^2 e^6}{3(4\pi\epsilon_0)^3 m_e^2 c^3 \hbar}$$

If v is taken from Maxwell-Boltzmann distribution, $v_e = \sqrt{3k_B T / m_e}$

$$P_{\text{Brem}} = 0,5 \times 10^{-36} z^2 n n_e (k_B T)^{1/2} \text{ W/m}^3$$

where $k_B T$ is in keV.

The reaction rate for fusion is $n_1 n_2 \langle \sigma v \rangle$, if there is only one kind of ions, $n_1 n_2 = \frac{1}{2} n^2$. If one plot the power density of the reaction with the Bremsstrahlung:



FOR D-T reaction the temperature at which fusion exceed bremsstrahlung loss is of the order of ≈ 4 keV.

For D-D it is about 4 keV
 \Rightarrow D-T is a superior fuel!

Energy loss by Bremsstrahlung is $\propto z^2$; each nucleus different from H have a large bremsstrahlung and a smaller reaction rate.

Other radiation losses, including synchrotron radiation from charged particle orbiting about magnetic field lines can be neglected.

Fusion reactor will have a net ENERGY GAIN if the ^{energy} released in reactions exceeds the radiation loss and the original energy invested in heating the plasma.

If reactor operates at $T > 4$ keV, at 10 keV, the D-T fusion gain is greater ⁽¹²¹⁾ than the radiation loss.

The energy released per unit volume from fusion reaction in plasma is

$$E_f = \frac{1}{4} n^2 \langle \sigma v \rangle Q \tau$$

where $n_D = n_T = \frac{1}{2} n$, $n = n_e$, Q is the energy released in the reaction ($Q = 17.6$ MeV for D-T), and τ is the length of time the plasma is confined.

The thermal energy per unit volume needed to raise the ions and e^- to temperature T is $\frac{3}{2} n k_B T$ (for ions) and $\frac{3}{2} n_e k_B T$ (for e^-). With $n = n_e$

$$E_{th} = 3 n k_B T$$

Reactor has an energy gain if $E_f > E_{th}$

$$\frac{1}{4} n^2 \langle \sigma v \rangle Q \tau \geq 3 n k_B T$$

$$n \tau \geq \frac{12 k_B T}{\langle \sigma v \rangle Q} \quad \equiv \text{Lawson criterion}$$

For D-T $T \approx 10$ keV, $\langle \sigma v \rangle_{DT} \approx 10^{-22} \text{ m}^3/\text{s}$, $n \tau > 10^{10} \frac{\text{s}}{\text{m}^3}$

For D-D $k_B T = 100$ keV $\langle \sigma v \rangle_{DD} \approx 0.5 \cdot 10^{-22} \text{ m}^3/\text{s}$, $n \tau > 10^{22} \text{ s/m}^3$

\Rightarrow For D-D we have that: to increase the density of ions 10^{12} times to get the same gain energy.

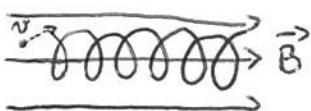
BASIC REACTOR TYPES AND HOW THEY MEET LAWSON CRITERION

(A) UNIFORM MAGNETIC FIELD

The simplest magnetic confinement is a uniform magnetic field

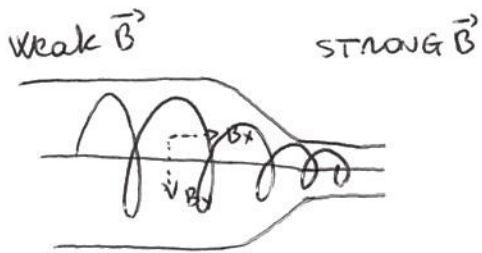
If a ptc moves in the field it can be confined by a simple field. At a certain point the

wire will end. To overcome such problem we can think



To use a non-uniform magnetic field

B) NON-UNIFORM MAGNETIC FIELD



$$\vec{v} = -v_z \hat{k} + v_x \hat{i}$$

$$\vec{B} = B_x \hat{i} - B_y \hat{j}$$

$$-v_z \hat{k} \times B_y \hat{j} - B_y \hat{j} \times v_x \hat{i}$$

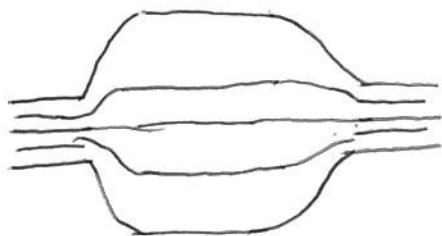
$$-v_z \hat{k} \times B_y \hat{j} = -v_z B_y \hat{i} - v_x B_y \hat{z}$$

$$\vec{F} = q \vec{v} \times \vec{B} = q [-v_z \hat{k} + v_x \hat{i}] \times [B_x \hat{i} - B_y \hat{j}] =$$

$$= q \left[(-v_z B_y) \hat{i} + (-v_z B_x) \hat{j} + (-v_x B_y) \hat{z} \right]$$

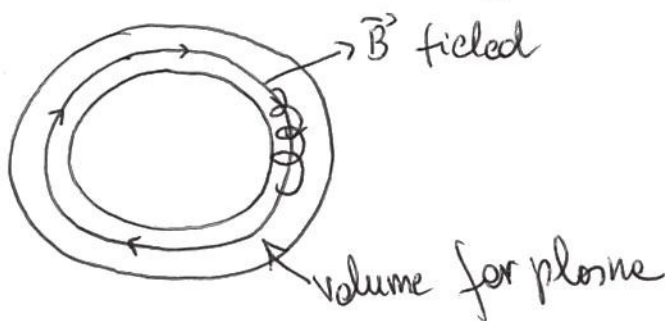
⇒ For a negative v_x the pte is pushed back!

Similarly it is possible to build a magnetic bottle



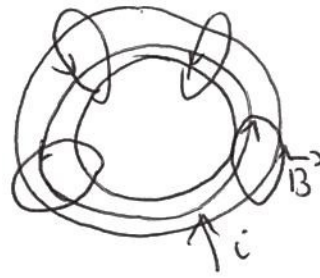
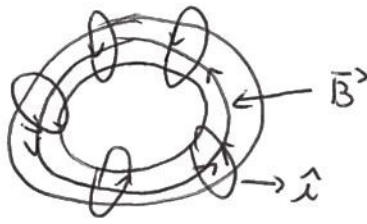
And pte can go back and forth.

The idea which is resulting to be the best comes from Russia (1968)



Like that there is a problem, only with a toroidal field, the problem is that the magnetic field is not constant with the radius

⇒ Combination of 2 magnetic field 1 toroidal and 1 poloidal (123)



The combination of the 2 is at the basis of the so called TOKAMAK. (TORoidal naja KAmera MAgnetnaja Katushka)
Toroidal Camera Magnetic coil

At present exists some "Experimental" tokamak ~~etc~~ accelerators such as ASDEX (Auxiliary Symmetric Divertor Experiment) in Max-Planck Institute für Plasmaphysik in Garching exist.

In 2025 it is foreseen the start of an international program called ITER (see slides).

A second idea to confine the plasma which ~~was~~ has been used is called "Inertial confinement fusion".

It is possible for example to have a tiny pellet containing deuterium and tritium suddenly struck with an intense laser pulse that both heats the pellet and compresses it to high density.

The goal of this technique is to achieve densities and temperatures that are high enough that fusion occur before the pellet expands and blows apart.

We can make a crude estimate for such device, considering that the time necessary for the compressed pellet to blow apart will be determined by the speed of propagation of mechanical waves in the medium.

This velocity is \approx to the thermal speed of ptc in the medium.

(→ i.e. for solid at room temperature $v \approx 10^3$ m/s)

At $kT \approx 10 \text{ keV}$ the mean thermal speed is $\approx 10^6 \text{ m/s}$. If the pellet has a diameter of $0.1 \div 1 \text{ mm}$, the pellet blow apart in $10^{-9} \div 10^{-10} \text{ s}$. Applying Lawson's criterion for D-T, we should need a density of $\approx 10^{29} - 10^{30} / \text{m}^3$ which is 2 order of magnitude greater than the ordinary liquid.

To heat a spherical pellet of 1 mm diameter to a mean thermal energy of 10 keV , the energy that must be supplied is

$$E_{\text{th}} \approx \frac{4}{3} \pi (0.5 \text{ mm})^3 \times 10^{29} \text{ m}^{-3} \times 10^4 \text{ eV} \approx 10^5 \text{ J}$$

\Rightarrow We must have an energy of $\approx 10^5 \text{ J}$ for $10^{-9} \text{ s} \Rightarrow 10^{14} \text{ W!}$

This estimate does not take into account that a portion of energy will be absorbed by the surface ^{and} that the ^{energy needed for} laser energy is not ^{all} converted to radiation energy. These 2 contributions led to a power of 10^{17} W which can be compared with eg. the entire electrical capacity of the USA which is of the order of 10^{12} W!

This means that to run an inertial confinement reactor the Lawson law should be exceeded considerably.

To do so the laser-driven fusion should follow these steps:

- ① A pellet is injected into the machine and struck from many directions by an intense laser pulse.
- ② The outer layers of the solid pellet is vaporized and form a plume which absorb the laser radiation
- ③ the plasma is unconfined and blow-off: by Newton's 3rd law a compressed shock wave ^{core of the} hits the remaining pellet
- ④ An additional thermonuclear reaction can occur inside the core near the center of the pellet.